

**Calculation the Cross Sections of  ${}^9\text{B}(n,\alpha){}^6\text{Li}$  reaction by using the  
reciprocity theory for the first excited state**

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**Abstract**

In this study light elements  ${}^6\text{Li}$  ,  ${}^9\text{B}$  for  ${}^9\text{B}(n,\alpha){}^6\text{Li}$  reaction as well as  $\alpha$ -particle energy from 6.84 MeV to 9.94 MeV are used according to the available data of reaction cross sections with threshold energy (3.9537MeV). The more recent cross sections data of  ${}^6\text{Li}(\alpha,n){}^9\text{B}$  reaction is reproduced in fine steps (50keV) in the specified energy range , as well as cross section ( $\alpha,n$ ) values were derived from the published data of ( $n,\alpha$ ) as a function of energy in the same fine energy steps by using the principle inverse reaction . This calculation involves only the first excited state of  ${}^6\text{Li}$  ,  ${}^9\text{B}$  in the reactions  ${}^6\text{Li}(\alpha,n){}^9\text{B}$  and  ${}^9\text{B}(n,\alpha){}^6\text{Li}$  .

**الخلاصة**

في هذه الدراسة اعيد حساب المقاطع العرضية للنوى الخفيفة ( ${}^6\text{Li}$  ,  ${}^9\text{B}$ ) للتفاعل  ${}^9\text{B}(n,\alpha){}^6\text{Li}$  للبيانات المتوفرة في الادبيات العالمية وللمدى الطاقى من (6.84) MeV الى (9.94) MeV وبطاقة عتبه مقدارها (3.9537MeV) كدالة للمقاطع العرضية . بأستخدام نظرية التعاكس اذ اشتقت معادلة لحساب المقاطع العرضية لتفاعل  ${}^9\text{B}(n,\alpha){}^6\text{Li}$  وللمستوى المتهيج الاول وذلك بالاعتماد على المقاطع العرضية لتفاعل  ${}^6\text{Li}(\alpha,n){}^9\text{B}$  ومن ثم الحصول على معادلة للرسم البياني من خلال استخدام برامج الحاسوب ( Matlab-6.5 ) . تم رسم وجدولة النتائج بالاضافة الى مناقشة النتائج .

### Introduction

The interaction of particles with matter is described in terms of quantities known as cross sections which is defined in the following way [1]. Consider a thin target of area (a) and thickness (X) containing (N) atoms per unit volume, placed in a uniform mono-directional beam of incident particles (neutrons for example of intensity  $I_0$ , which strikes the entire target normal to its surface as shown in fig.(1). It is found that the rate at which interactions occur within the target is proportional to the beam intensity and to

### Reciprocity Theory

If the cross-sections of the reaction  $A(\alpha,n)B$  is measured as a functions of  $T_\alpha$  ( $T_\alpha$  = Kinetic energy of  $\alpha$ -particle) the cross-sections of the inverse reaction  $B(n,\alpha)A$  can be calculated as a function of  $T_n$  ( $T_n$  = Kinetic energy of neutron) using the reciprocity theorem [3] which states that :

$$\frac{\sigma_{(\alpha,n)}}{\sigma_{(\alpha,n)} \lambda_\alpha^2} = \frac{\sigma_{(n,\alpha)}}{g_{(n,\alpha)} \lambda_n^2} \quad \text{---(1-3)}$$

Where  $\sigma_{(\alpha,n)}$  and  $\sigma_{(n,\alpha)}$  represent cross-sections of  $(\alpha,n)$  and  $(n,\alpha)$  reactions respectively ,  $g$  is a statistical factor and  $\lambda$  is the de-Broglie wave length divided by  $2\pi$  and is given by

$$\lambda = \frac{h}{MV} \quad \text{----(1-4)}$$

Where  $h$  is Dirac constant ( $h/2\pi$ ),  $h$  is plank constant ,  $M$  and  $v$  are mass and velocity of  $\alpha$  or  $n$  particle .

the atom density, area and thickness of the target. Summarizing this experimental result by an equation, we define the interaction rate

$$(\text{in the entire target}) = \sigma I N a X \quad \text{--- (1-1)}$$

Where the proportionality constant  $\sigma$  is known as the cross section , Thus

$$\sigma = \text{interaction rate} / I N a X \quad \text{---- (1-2)}$$

As  $N a X$  is equal to the total number of atoms in the target, it follow that  $\sigma$  is the interaction rate per atom in the target per unit intensity of the incident beam [2] .

From eq.(1-4),we have

$$\lambda^2 = \frac{h^2}{2MT} \quad \text{-----(1-5)}$$

The statistical g-factors are gives by[3]

$$g_{(\alpha,n)} = \frac{2J_c + 1}{(2I_A + 1)(2I_\alpha + 1)} \quad \text{---(1-6)}$$

And

$$g_{(n,\alpha)} = \frac{2J_c + 1}{(2I_B + 1)(2I_n + 1)} \quad \text{----(1-7)}$$

The conservation law of the momentum implies that:

$$I_A + I_\alpha = J_c = I_B + I_n \quad \text{----(1-8)}$$

And

$$\pi_A \pi_\alpha (-1)^{l_\alpha} = \pi_c = \pi_B \pi_n (-1)^{l_n} \quad \text{----(1-9)}$$

$J_c$  and  $\pi_c$  are total angular momentum and parity of the compound nucleus .

$I_A$  and  $\pi_A$  are total angular momentum and parity of nucleus A.

$I_B$  and  $\pi_B$  are total angular momentum and parity of nucleus B.

$I_\alpha$  and  $\pi_\alpha$  are total angular momentum and parity of  $\alpha$ -particle.

$I_n$  and  $\pi_n$  are total angular momentum and parity of neutron .

$$\pi_\alpha = \pi_n = +1 \text{ -----(1-10)}$$

$$I_\alpha = s_\alpha + \ell_\alpha \text{ -----(1-11)}$$

Where  $I_\alpha$  is the total angular momentum of alpha particle.

$s_\alpha$  is spin of  $\alpha$ -particle = 0

$\ell_\alpha$  is the orbital angular momentum of  $\alpha$ -particle.

And

$$I_n = s_n + \ell_n \text{ -----(1-12)}$$

Where  $I_n$  is the total angular momentum of the neutron

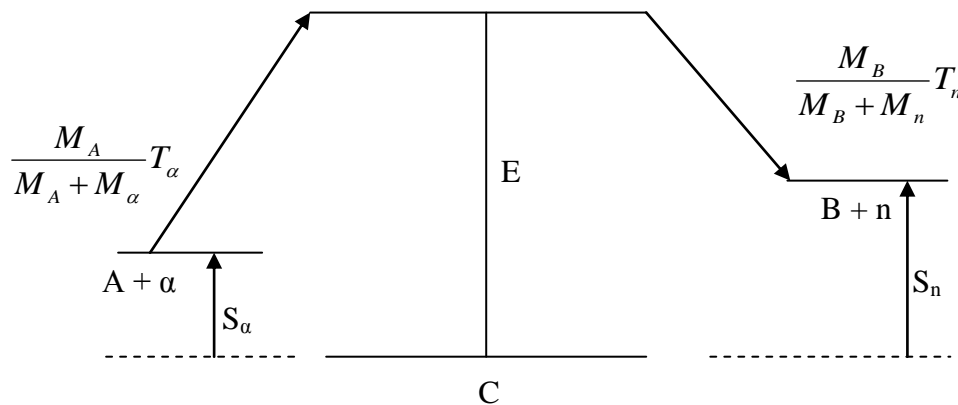
$s_n$  is spin of neutron = 1/2

$\ell_n$  is the orbital angular momentum of neutron .

From eq.(1-8),we have:

$$| J_c - I_\alpha | \leq \ell_\alpha \leq J_c + I_\alpha \text{ -----(1-13)}$$

And



Schematic diagram of the reactions

and as the Q-value of the reaction  $A(\alpha,n)B$  is given by :

$$Q = 931.5 [ M_A + M_\alpha - M_B - M_n ] \text{ ---(1-18)}$$

$$| J_c - I_n | \leq \ell_n \leq J_c + I_n \text{ -----(1-14)}$$

The reactions  $A(\alpha,n)B$  and  $B(n,\alpha)A$  can be represented with the compound nucleus C as in the following schematic diagram. It is clear that there are some important and useful relations between the kinetic energies of the neutron and alpha particle. One can calculate the separation energies of  $\alpha$ -particle ( $S_\alpha$ ) and neutron ( $S_n$ ) using the following relations:

$S_\alpha$  and  $S_n$  are separation energies of  $\alpha$  and n from C. Then

$$E = S_\alpha + \frac{M_A}{M_A + M_\alpha} T_\alpha \text{ ---(1-15a)}$$

$$E = S_n + \frac{M_B}{M_B + M_n} T_n \text{ ---(1-15b)}$$

With

$$S_\alpha = 931.5 [ M_A + M_\alpha - M_c ] \text{ ----(1-16)}$$

$$S_n = 931.5 [ M_B + M_n - M_c ] \text{ ----(1-17)}$$

Combining (1-15a) , (1-15b) , (1-16) and (1-17)

Then

$$Q = \frac{M_B}{M_B + M_n} T_n - \frac{M_A}{M_A + M_\alpha} T_\alpha \text{ ----(1-19)}$$

Or :

$$T_n = \frac{M_B + M_n}{M_B} \left[ \frac{M_A}{M_A + M_\alpha} T_\alpha + Q \right] \quad \text{---(1-20)}$$

The threshold energy  $E_{th}$  is given by :

$$E_{th} = -Q \frac{M_A + M_\alpha}{M_A} \quad \text{---(1-21a)} \quad \text{Or}$$

$$Q = -\frac{M_A}{M_A + M_\alpha} E_{th} \quad \text{---(1-21b)}$$

Then

### Results and Discussion

The cross section of  $(\alpha, n)$  reactions for the elements  ${}^6\text{Li}$  and  ${}^9\text{B}$  of  ${}^6\text{Li}(\alpha, n){}^9\text{B}$  reaction available in the literature[4] , have been taken and re-plotted for a defined energy level as shown in Fig.(2). These plots were analyzed using the Matlab computer program to obtain the cross sections for the selected energies and we get equation for these data as follow:

$$y = 0.0022 * x^{10} - 0.18 * x^9 + 6.4 * x^8 - 1.4e+2 * x^7 + 1.9e+3 * x^6 - 1.9e+4 * x^5 + 1.2e+5 * x^4 - 5.6e+5 * x^3 + 1.7e+6 * x^2 - 2.9e+6 * x + 2.3e+6$$

The atomic mass of elements and isotopes mentioned in this study have been taken from the latest nuclear wallet cards released by the National Nuclear Data Center(NNDC)[5] and the energy level, parity and spin scheme of isotopes from [6].

By using the reciprocity theory we derive the mathematical formula for  ${}^9\text{B}(n, \alpha){}^6\text{Li}$  reaction for first excited state:

$$\sigma_{n, \alpha} = 3.720 \frac{T_\alpha}{T_n} \sigma_{\alpha, n}$$

$$T_n = \frac{M_B + M_n}{M_B} \times \frac{M_A}{M_A + M_\alpha} (T_\alpha - E_{th}) \quad \text{---(1-22)}$$

Thus eq.(1-3) can be written as follows :

$$\sigma_{(n, \alpha)} = \frac{g_{(n, \alpha)} M_\alpha T_\alpha}{g_{(\alpha, n)} M_n T_n} \sigma_{(\alpha, n)} \quad \text{---(1-23)}$$

It is clear from this equation that the cross sections of reverse reaction are related by a variable parameters which can be calculated if the nuclear characteristics of the reactions are known.

By using semi empirical formula the evaluated cross sections as a function of neutron energy from (1.9269)MeV to (4.0298)MeV of present work are listed in table (1). From these data which were plotted, we got the mathematical equation representing the cross sections distribution in the indicated range of neutron energy Fig.(3) and percentage error ( $\pm 0.1819$ ) barn as follow :

$$Y = 0.011X^{10} + 0.038X^9 - 0.044X^8 - 0.19X^7 + 0.038X^6 + 0.24X^5 - 0.088X^4 - 0.099X^3 + 0.086X^2 + 0.19X + 0.44$$

From cross sections of  ${}^6\text{Li}(\alpha, n){}^9\text{B}$  we calculated the neutron yield [7][8] by using stopping power [9] from the Zeigler formula (SRIM-2008)[10]. We are plotted in figure(4).

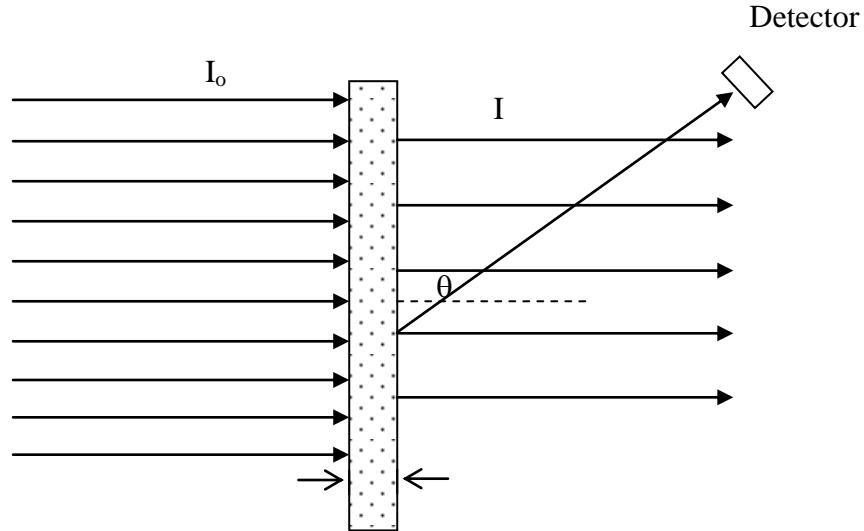


Figure (1): A schematic diagram illustrating the definition of total cross section in terms of the reduction of intensity[1].

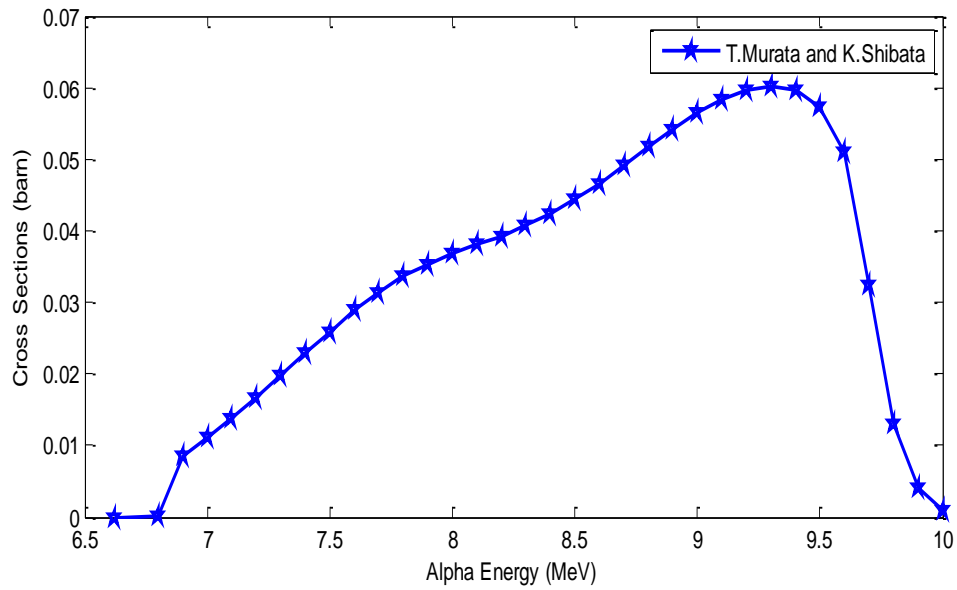


Fig.( 2): Cross sections of  ${}^6\text{Li}(\alpha,n){}^9\text{B}$  Reaction[4]

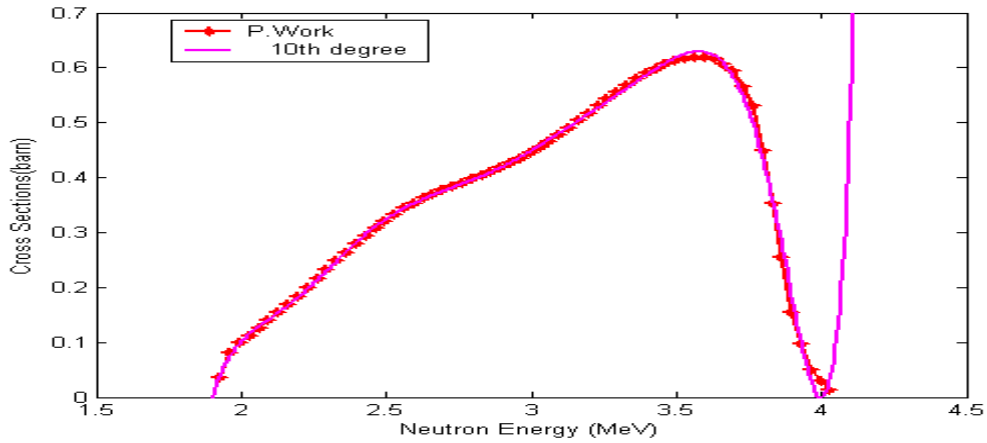


Fig.(3): Cross sections of  ${}^9\text{B}(n,\alpha){}^6\text{Li}$  Reaction of P.Work

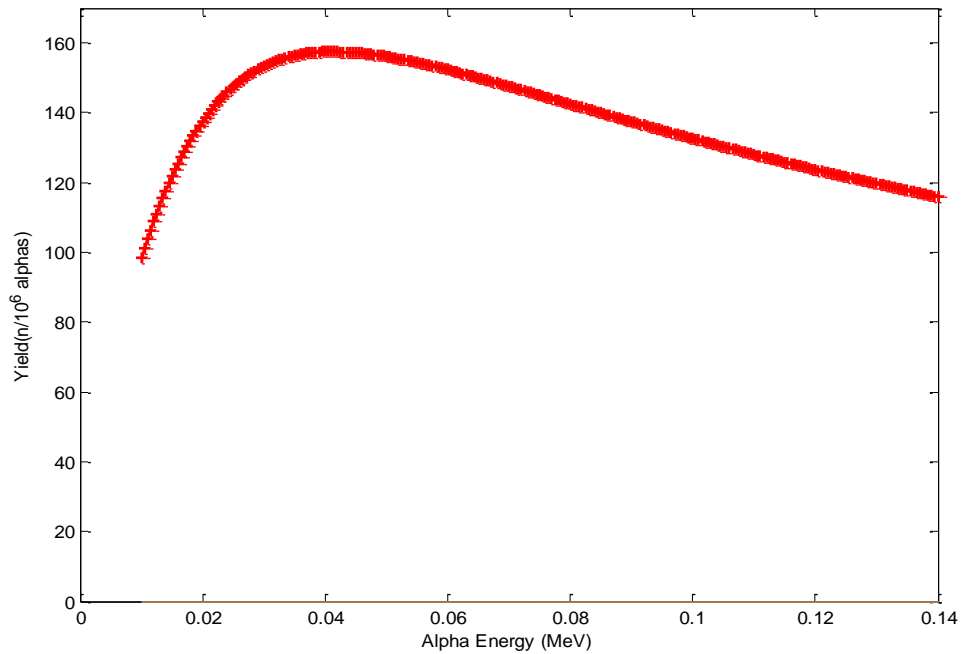


Fig.(4) :Neutron Yield for  ${}^6\text{Li}(\alpha,n){}^9\text{B}$  reaction

Table (1):The cross sections of  ${}^9\text{B}(n,\alpha){}^6\text{Li}$  Reaction as a function of neutron energy present work.

neutron - energy (MeV)	X- sections (barn) P.Work	neutron - energy (MeV)	X- sections (barn) P.Work	neutron - energy (MeV)	X- sections (barn) P.Work
1.9269	0.0359	2.7280	0.3843	3.5291	0.6163
1.9603	0.0802	2.7614	0.3908	3.5625	0.6191
1.9937	0.0993	2.7948	0.3973	3.5959	0.6182
2.0270	0.1121	2.8282	0.4039	3.6293	0.6162
2.0604	0.1255	2.8615	0.4110	3.6627	0.6062
2.0938	0.1392	2.8949	0.4183	3.6960	0.5941
2.1272	0.1537	2.9283	0.4266	3.7294	0.5652
2.1606	0.1685	2.9617	0.4352	3.7628	0.5321
2.1939	0.1840	2.9951	0.4451	3.7962	0.4491

2.2273	0.1996	3.0284	0.4553	3.8296	0.3536
2.2607	0.2157	3.0618	0.4667	3.8629	0.2544
2.2941	0.2319	3.0952	0.4784	3.8963	0.1544
2.3275	0.2480	3.1286	0.4910	3.9297	0.0972
2.3608	0.2640	3.162	0.5039	3.9631	0.0508
2.3942	0.2792	3.1953	0.5172	3.9965	0.0292
2.4276	0.2943	3.2287	0.5306	4.0298	0.0138
2.4610	0.3080	3.2621	0.5437	----	----
2.4944	0.3213	3.2955	0.5567	----	----
2.5277	0.3329	3.3289	0.5686	----	----
2.5611	0.3441	3.3622	0.5802	----	----
2.5945	0.3535	3.3956	0.5901	----	----
2.6279	0.3625	3.429	0.5996	----	----
2.6613	0.3702	3.4624	0.6065	----	----
2.6946	0.3776	3.4958	0.6128	----	----

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